Consideration in Pre-Licensing Reviews of New Reactor Technologies

ORNL MSR Workshop 2016 Meeting

October 4-5, 2016

Oak Ridge National Laboratories

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Part 1:

CNSC Pre-Licensing Vendor Design Review (VDR) Process

VDR - A tool for reactor vendors

- To determine whether the vendor is ready for potential deployment in Canada
- A proven and standardized process to evaluate, in principle, whether there are fundamental barriers to licensing the vendor's reactor design in Canada
- The process should not be triggered unless the vendor's conceptual design is essentially complete and the basic engineering program has begun (design requirements being established)
- Outcomes of the process helps the vendor have discussions with potential future licensees interested in their technology

A Pre-licensing VDR is not a licensing discussion

It is a technical conversation between the CNSC and the vendor

Process is optional and not a prerequisite to licensing

Very Important Point...

The primary focus of a VDR is on the vendor's integrated management system processes used to develop a high quality design configuration. For example:

- Management of design, safety analysis, procurement
- Design methods (e.g. safety classification, application of codes and standards, human factors engineering)
- Safety analysis approaches and processes
- Program for collecting and integrating OPEX, R&D, other technical information

The outcome (a reactor's configuration) reflects the quality of these processes



- A VDR enables vendors and utilities to communicate, identify and address regulatory issues early enough so that delays in licensing and facility construction, can be minimized:
 - Higher quality licence applications
 - Efficient and effective licensing process
 - Assists decision makers in quantifying project risks (informing cost and schedule estimates)

Identify and resolve key issues before build - reducing cost and time risks, and ensuring public safety

Optional, standardized and technologyneutral process

CNSC uses a managed process:

- To evaluate, in principle, whether there are fundamental barriers to licensing the vendor's reactor design in Canada
- To ensure each vendor receives a fair and consistent review
- to standardize review topics and drive the review using a combination of documented internal work instructions and specialist expert judgement
- With schedule flexibility, within reason, to take into account a vendor's desired submission schedule

The outcome of the review process is <u>not</u> a detailed review of the entire design – It is a broad sample of key safety related topics



Process is documented in GD-385

- GD-385 Pre-licensing Review of a Vendor's Reactor Design, May 2012
 - Preserves vendor proprietary information while giving the public information through an Executive Summary
 - The review is solely intended to provide early feedback on the acceptability of selected aspects of a nuclear power plant design based on Canadian regulatory requirements and CNSC expectations
 - Is not certification of a design
 - Does not fetter the Commission in the licensing process

The CNSC will undertake a far more detailed review of the design at the time of review of a licence application for a specific site

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Relationship between Pre-licensing VDR and an eventual specific site project

- The results from the VDR process can be used to inform licensing activities
- Assuming the vendor shares results with the interested utility, the
 utility can shape their own licensing submissions with information
 obtained from the VDR process (but that information would then
 become part of the public process)
- Understanding the results of the VDR process can help a utility understand where project risks can emerge, e.g.:
 - Where the design may need adjustment to meet requirements
 - Where extra utility scrutiny over the vendor may be needed

Remember: A VDR is a with the Vendor. Licensing is with a Licensee (i.e. user of the vendor's technology)



Three-phases increasing review depth

- Phase 1: approximately 5,000 hours staff time (1 year to perform)
 - Does vendor design <u>intent</u> (i.e. in design processes) show an understanding of Canadian requirements? (examination of 19 Focus Areas)
 - does vendor understand regulatory language in Canada?
- Phase 2: approximately 10,000 hours staff time (18 months to 2 years to perform)
 - Phase 1 follow-up and assessment of the design for fundamental barriers to licensing in the 19 Focus Areas
 - is vendor addressing Canadian design and safety analysis requirements in specific aspects of the design? (i.e. Are outcomes reflecting the quality of the vendor's processes?)
- Phase 3: scope and depth requested by vendor (time varies)
 - Follow-up on review areas based on Phase 1 and 2 outcomes



Overview of Focus Areas used in Phases 1 and 2

	1 General NPP description - defence-in-depth, safety goals and objectives, and dose acceptance criteria	11 Pressure boundary design
	2 Classification of systems, structures & components	12 Fire protection
	3 Reactor core nuclear design (e.g. core physics)	13 Radiation protection
	4 Fuel design and qualification	14 Out-of-core criticality
	5 Control system and facilities (main control systems,	15A Robustness
	instrumentation and control, control facilities, emergency power systems)	15B Security and Cyber Security
		15C Safeguards
	6 Means of reactor shutdown	16 Vendor research and development program
	7 Emergency core cooling and emergency heat removal systems	17 Management system of design process and quality assurance in design and safety analysis
	8 Containment and safety important civil structures	18 Human factors
	9 Mitigation of Design Extension Conditions	19 Incorporation of decommissioning into design considerations
	10 Safety Analysis (Deterministic Safety Analysis, Probabilistic Safety Analysis, Internal and External Hazards)	



Entering the Pre-licensing VDR process

- The process should not be triggered by a vendor unless:
 - Phase 1: the conceptual design is essentially complete and the basic engineering program has begun (design requirements and safety specifications being established)
 - Management system processes are being implemented
 - Phase 2: generic safety analysis report development is underway
 - Management system processes for design and safety analysis are documented and being used
 - Design quality assurance processes are established and being used

Examples of CNSC Requirements That Can Already Be Applied to SMRs

- REGDOC-2.4.1, Deterministic Safety Analysis
- REGDOC-2.5.2, Design of Reactor Facilities: Nuclear Power Plants (for larger SMRs)
- RD-367, Design of Small Reactor Facilities (for smaller SMRs)
- REGDOC-2.3.2, Accident Management, version 2
- REGDOC-2.10.1, Nuclear Emergency Preparedness and Response

All address the use of the graded approach and are written to permit use of (supported) judgement

"Point" to where your design requirements address each REGDOC clause

- If referring to a Design Certification Document (DCD) or similar document, this tells CNSC where the requirement is addressed
- Information should include references to applicable codes and standards
- Are there any Fukushima or OPEX lessons applicable to this Focus Area?
 - if so, how are they being addressed?
- If using codes and standards from outside Canada the should vendor identify gaps between their adopted standard and those used in Canada.

Identify any "novel features" and outstanding R&D for the focus area

- Novel features, by nature, are not yet proven
 - examples:
 - New core configuration / fuel type
 - passive behaviour of a preventive or mitigating system
- What is the path forward to show the novel feature will meet requirements?
- Give an overview of R&D being undertaken for the novel feature(s) and identify outstanding work to be done



Part 2: Considerations in establishing a Pre-Licensing Process



 Need to understand in advance what the process will be used for later (e.g. in licensing)

Focus on the Capabilities/Capacities of the Vendor

- Does the vendor have integrated processes to manage design and safety analysis activities. E.g.
 - Is the R&D <u>program</u> quality being managed? (in Canada CSA Group N286-12 applies)
- In Pre-licensing discussions, design completeness is less important than progress being made to perform key design activities
- Vendor is generating credible basis information to support safety claims

Determination of "Proven" can be quite challenging

 All regulators looking at SMRs and Gen IV technologies are asking the question "what level of evidence is necessary to make the determination of 'proven enough' for:

Prototypical experiments	To collect specific scientific/ engineering information on (proof of concept)	Low state of proven-ness – risks and uncertainties are higher – additional safety & control measures needed
Demonstration reactor / First-of-a Kind	Demonstration of integrated components / systems and collection of OPEX to refine design for nth of a kind	Varying amounts of OPEX – proving in progress- varying risks and uncertainties to be addressed – some additional safety & control measures needed where uncertainties are high
"Nth"-of-a-Kind	Commercial operation – information used to improve operational performance	High state of proven-ness – uncertainties generally well understood and ongoing R&D supports management of uncertainties

'Proven' is both technical and process-driven (different technical specialist areas are involved in the assessment)

Summary

- There are challenges and pressures with regards to deployment of SMRs / advanced reactors
- Well established EA, licensing and VDR processes in Canada
- Regulatory framework is comprehensive
 - CNSC is reviewing their current regulatory framework and seeking feedback from stakeholders

 Vendor Design Reviews provide a framework for consideration of technologies, and in particular, novel approaches





Attachment: The Basics



The CNSC's role is regulatory oversight by:

- Ensuring regulatory requirements are clear
- Ensuring a balanced, efficient and transparent licensing process
- Confirming the licensee is meeting regulatory requirements and applying enforcement measures as necessary



Graded Approach in the Regulatory Framework – Reactor Facilities

 Methods used to establish stringency of the following commensurate with the level of risk posed by the reactor facility:

Design measures

Safety analyses

Provisions for operation

- Factors to be considered include:
 - reactor power, reactor safety characteristics, fuel design, source term
 - amount and enrichment of fissile and fissionable material
 - what the reactor is being utilized for
 - presence of high-energy sources and other radioactive and hazardous sources
 - safety design features
 - siting, proximity to populated areas

Requirements are not relaxed: Safety will not be compromised



Use of Alternative Approaches

CNSC will consider alternative approaches to requirements where:

- the alternative approach would result in an equivalent or superior level of safety
- the application of the requirements conflicts with other rules or requirements
- the application of the requirements would not serve the underlying purpose, or is not necessary to achieve the underlying purpose

Alternative approaches must be explained and supported with suitable information



Section 24(4) of the NSCA

No licence shall be issued, renewed, amended or replaced — and no authorization to transfer one given — unless, in the opinion of the Commission, the applicant:

- a) is qualified to carry on the activity that the licence will authorize the licensee to carry on
- b) will, in carrying on that activity, make adequate provision for the protection of the environment, the health and safety of persons and the maintenance of national security and measures required to implement international obligations to which Canada has agreed

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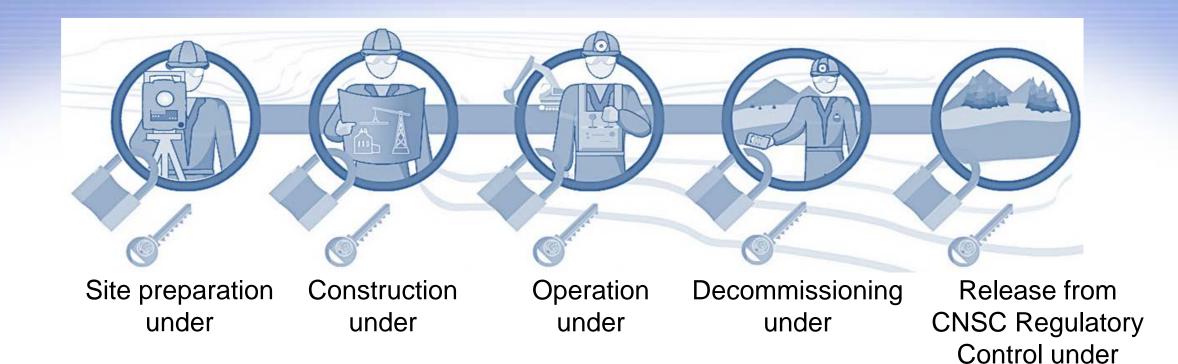
Licence to

Prepare Site

Licence to

Construct

Five stages (types of activities) in the lifecycle of a nuclear facility



Combined licenses are possible

Licence to

Operate

Licence to

Decommission

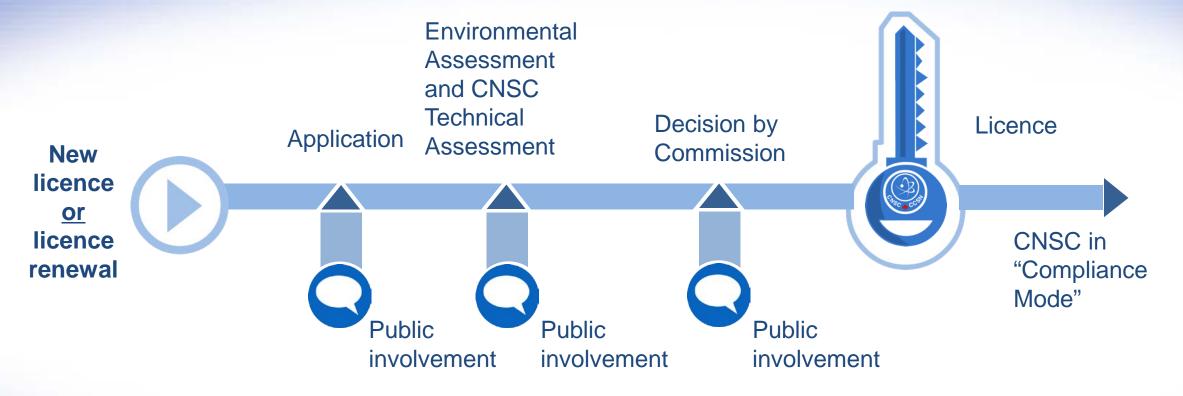
Licence to

Abandon



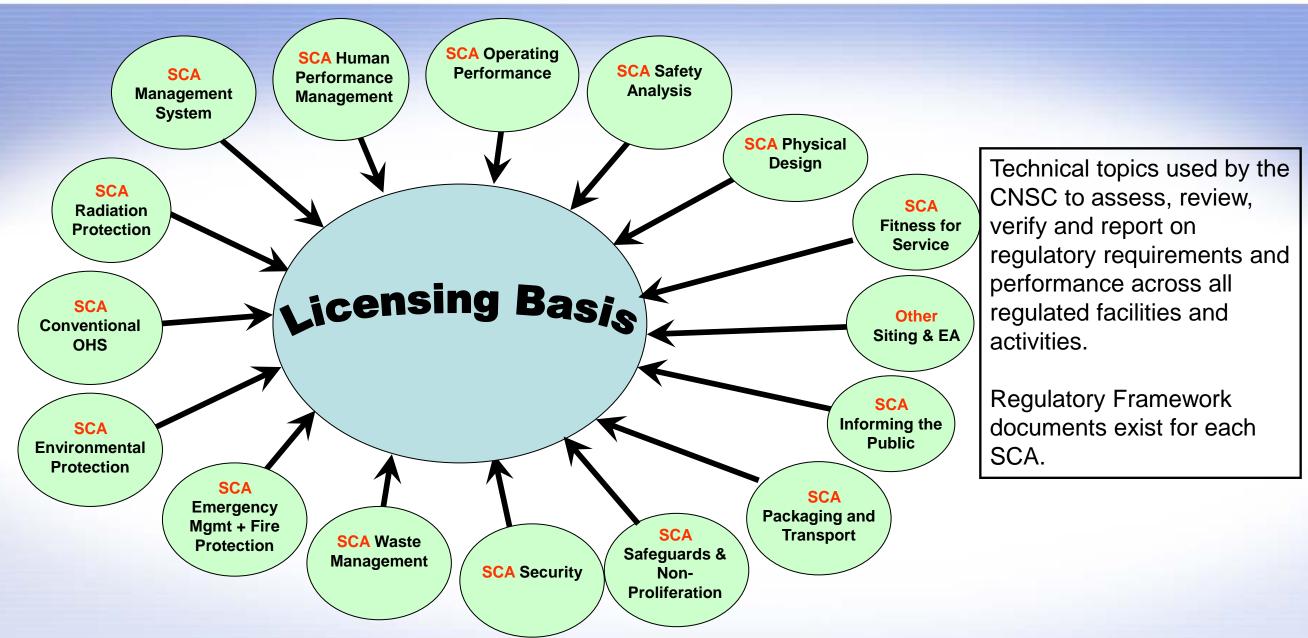
CNSC Licensing Process Overview

One process, regardless of facility size



Ongoing public involvement, Aboriginal consultation and environmental monitoring

Safety and Control Area (SCA) Framework





Canadian Environmental Assessment Act 2012: Environmental Assessments are the responsibility of the CNSC

- Before a licence to prepare a site can be issued, the environmental impact of the project must be considered for the lifecycle of the project
- Legislated timelines apply to EA and first licence (generally a Licence to Prepare Site)
 - CNSC has service standards for subsequent licences
- EA process is independent of facility size
- The province / territory may have involvement in the EA process jurisdiction dependent
- Other federal departments are involved in CEAA 2012 EAs



Challenges being presented to regulators

- New technologies can have just as many uncertainties as the first generation
 - Adding more new features over and above those tested in the past
 - How much of the original experimental evidence is valid/useful?
 - Commercial power reactor operating cycle ≠ cycle of experimental facilities
- Investors funding technology in smaller discrete steps
 - This influences the scope and depth of R&D at each phase of development, vendors looking for regulatory feedback
- Utilities under greater cost pressures
 - More aggressive plant performance including optimized maintenance and operation
 - Questioning rationales behind new regulatory requirements regulator needs to explain why those requirements are necessary