

2022 MSR Workshop



SAM FOR MSR SYSTEM ANALYSIS



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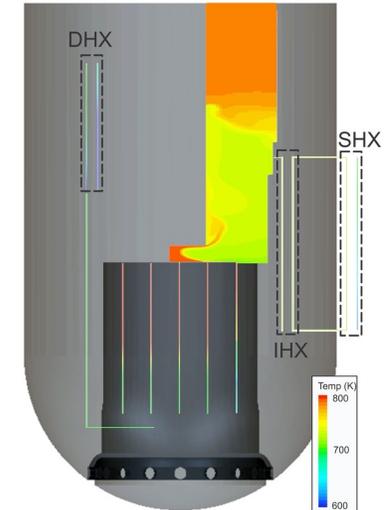
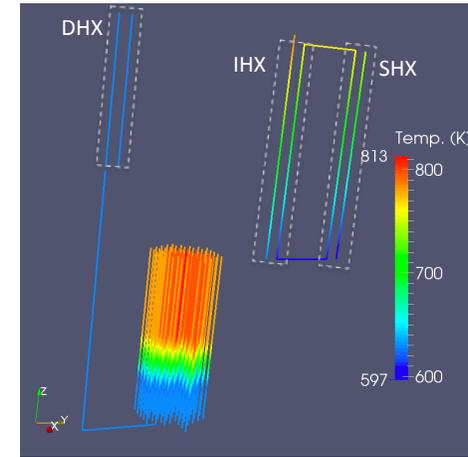
OCT. 11, 2022

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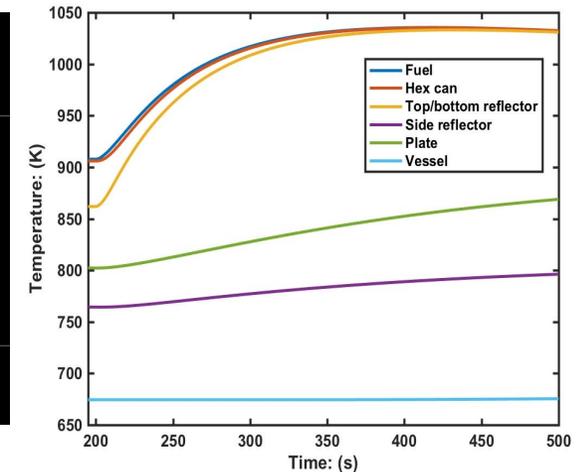
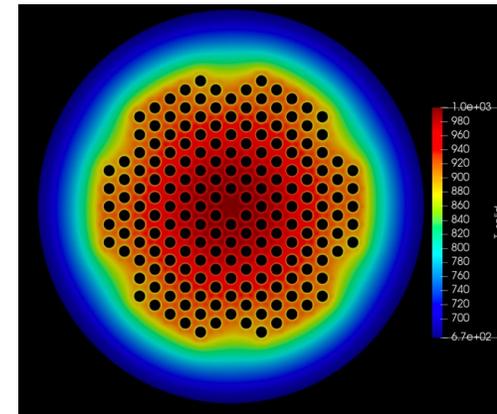
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SAM OVERVIEW

- A modern system analysis code for advanced non-LWR safety analysis.
 - Cover almost all non-LWR concepts.
- Advances in software engineering, numerical methods, and physical models (built-on MOOSE framework and its libraries);
- Advancements in loop system, core, and large volume modeling;
- Flexible multi-scale multi-physics integration with other MOOSE- or non-MOOSE-based tools.
- Part of BlueCRAB for confirmatory calculations of licensing applications at USNRC.
- 2019 R&D 100 Award.



Stand-alone and Coupled SAM and CFD code simulations of SFR



Transient multi-physics simulation of heat-pipe-cooled micro-reactor

What Are in Included SAM?

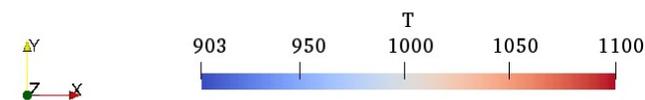
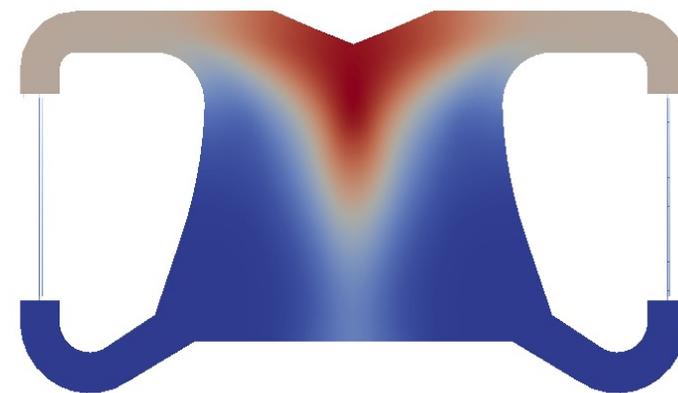
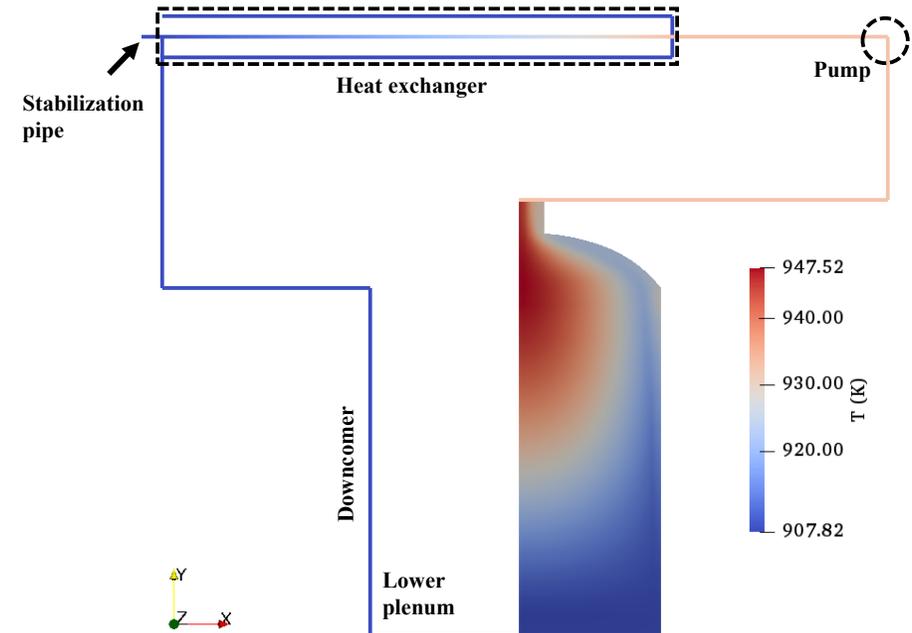
- Closely related to what it does

▪ “System computer codes must be able not only to simulate **several subsystems**, **many components** and their **couplings**, but also the **simultaneous occurrence** of various **phenomena** and **processes**.” – Salomon Levy

- Flow models (single-phase, 1-D, multi-D, ...)
- Heat transfer (convective, conduction, thermal radiation, ...)
- Reactor kinetics (point kinetics, decay heat, reactivity feedbacks)
- Special components (pump, heat exchanger, valves, heat pipes, fluid diode, liquid volume, ...)
- Special processes (mixing, thermal stratification, ...)
- Special models (fluid solidification, species transport, structure expansion, ...)
- Control & trip systems
- UQ integration
- Multi-scale multi-physics coupling

SAM CODE CAPABILITIES FOR MSR

- Relevant/Specific MSR capabilities:
 - Flexible input of fluid properties – include user defined ones.
 - General heat transfer correlations, applicable to molten salts, and allow user defined ones.
 - Flexible heat transfer modeling with multi-scale approach.
 - Reactor kinetics model for circulating fuel.
 - Flexible reactivity feedbacks.
 - Fission product (including delayed neutron precursor) transport models, including interaction with graphite, bubbles.
 - Tritium transport model.
 - Fluid solidification.



MSR TRANSIENT MODELING – SPECIES TRANSPORT

- A passive scalar particle transport model was developed to track the distributions of particles in the fluid

- Delayed neutron precursors
- Decay heat precursors
- Tritium
- Fissile fuel particles

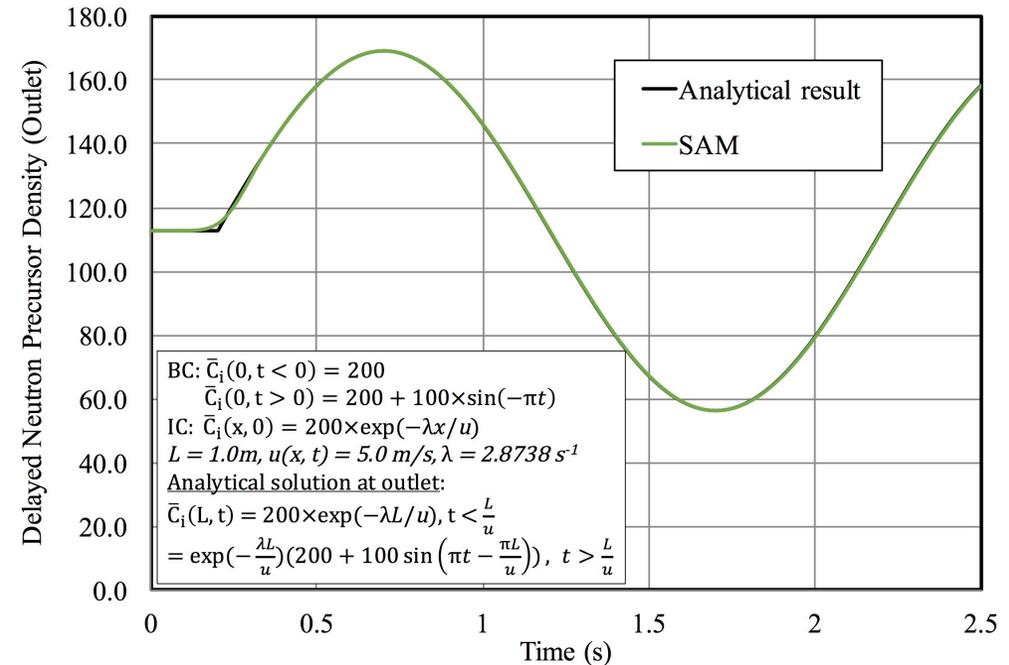
$$\frac{\partial \rho c_i}{\partial t} + \nabla (\rho u c_i) - \nabla (D_i \nabla \rho c_i) + \lambda_i \rho c_i = S_i$$

- Species Transport in Solid

- Diffusion,
- trapping, detrapping in tritium transport

- Species Transport at Fluid-Solid interface

- Convective mass transfer
- Solubility constraints: Henry's law, Sievert's law



Delayed-Neutron Precursor Density at 1-D Pipe Outlet
(Analytical Solutions vs. SAM Simulation)

MSR TRANSIENT MODELING – REACTOR KINETICS

Point-Kinetic model with delayed neutron precursor drifting

- In addition to the regular Point-Kinetic model, delayed neutron precursor drifting in the MSR system is considered.

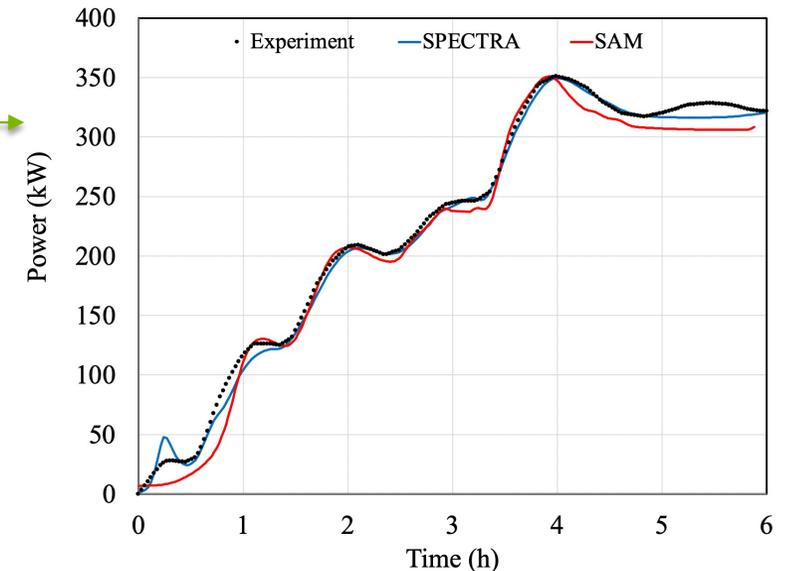
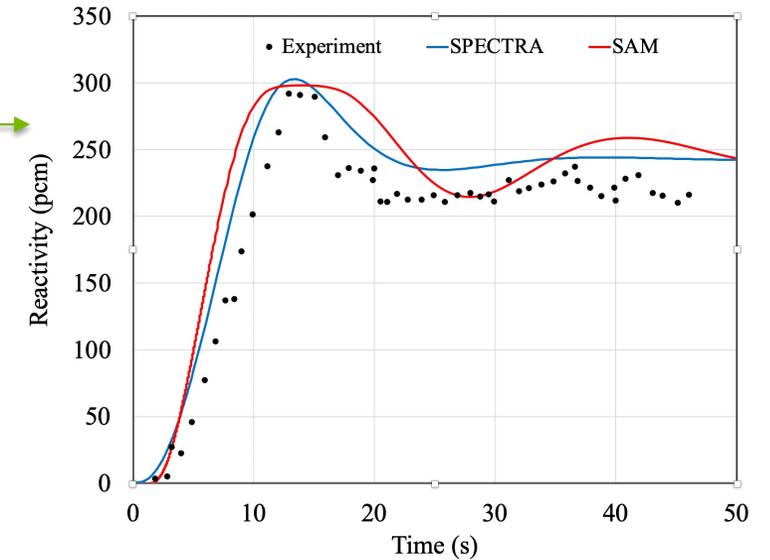
$$\frac{dn(t)}{dt} = \frac{\rho - \beta - \sum_i (\dot{c}_{in,i}(0) - \dot{c}_{out,i}(0)) \Lambda}{\Lambda} n(t) + \sum_i \lambda_i C_i$$

$$\frac{dC_i}{dt} = \frac{\beta_i}{\Lambda} n - \lambda_i C_i + (\dot{c}_{in,i} - \dot{c}_{out,i})$$

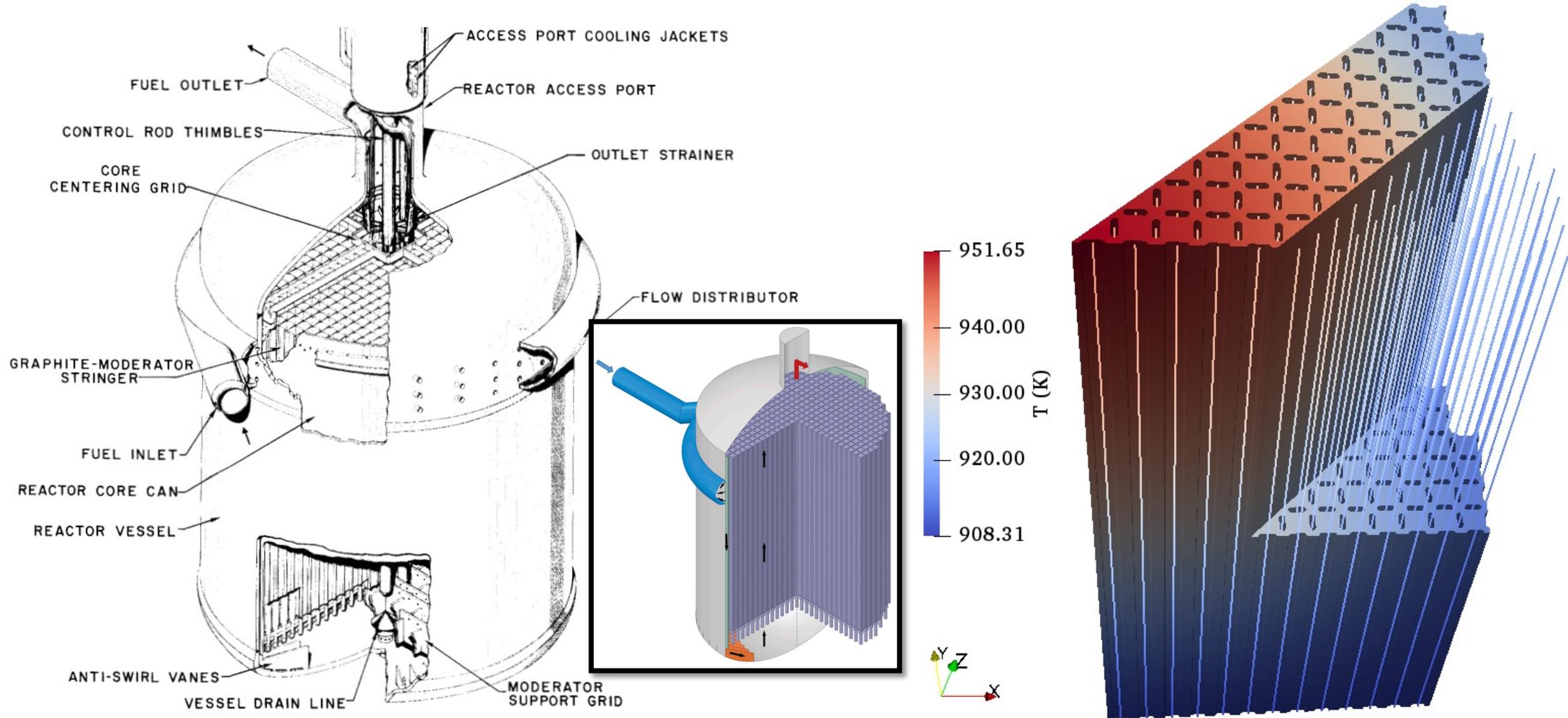
- Flexible reactivity feedback modeling
 - Coolant reactivity feedback
 - Radial expansion feedback
 - Axial expansion feedback
 - Fuel Doppler feedback
 - Structure temperature reactivity feedback

MSRE TRANSIENT BENCHMARK

- Activity supported under NEAMS MSR Application Drivers; ORNL provided significant amounts of newly recovered MSRE data.
- Use the valuable MSRE data to enhance and validate SAM for MSR (flowing fuel salt) transient modeling and simulation
 - Pump startup tests
 - Pump shutdown tests
 - Natural convection tests
 - Reactivity insertion tests
 - *Frequency domain tests*

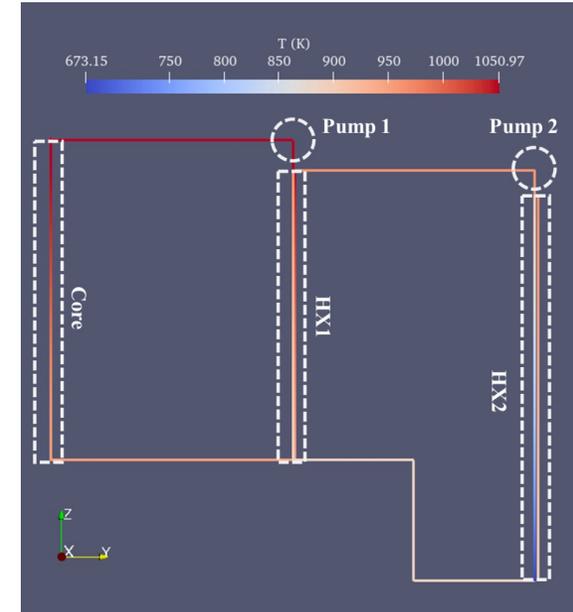


MSRE MODELING WITH COUPLED 3-D HEAT STRUCTURE AND 1-D CHANNELS

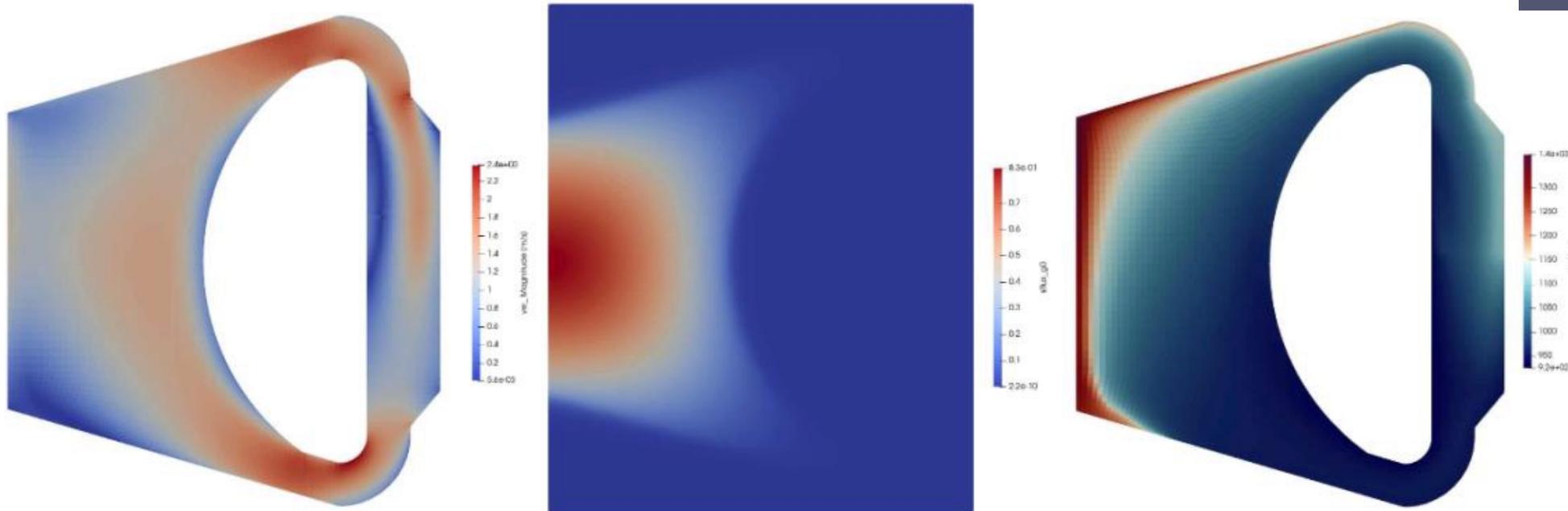


INTEGRATED MSFR DEMO

- Joint Argonne-INL efforts under NEAMS in FY22.
- Demonstration of a coupled SAM-Pronghorn-Griffin transient simulation of the MSFR
 - SAM: ex-core and the whole system; Pronghorn: core or primary loop
 - Griffin: core reactor physics
- Transient Modeled: MSFR primary loop pump coast down
 - 50% pump head drop, full pump trip

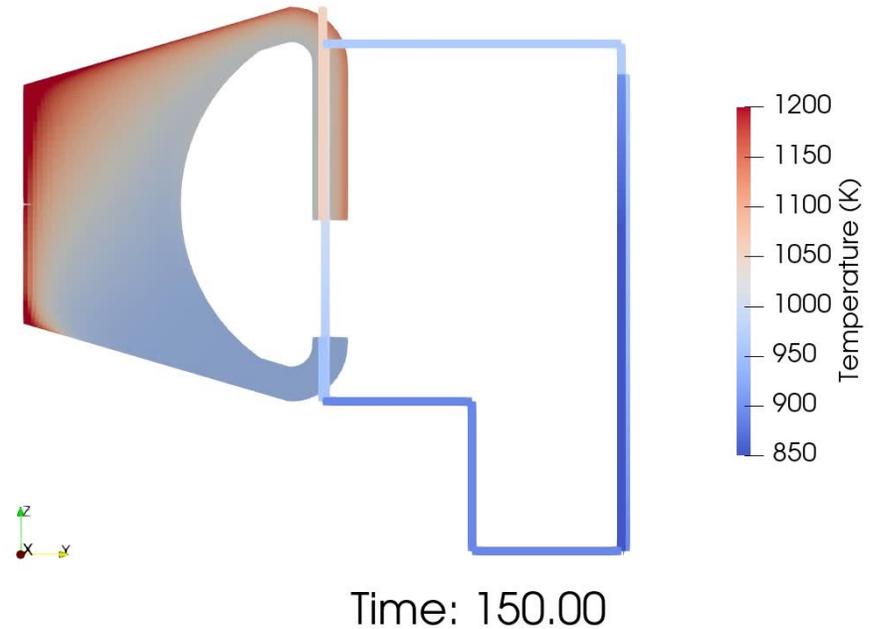
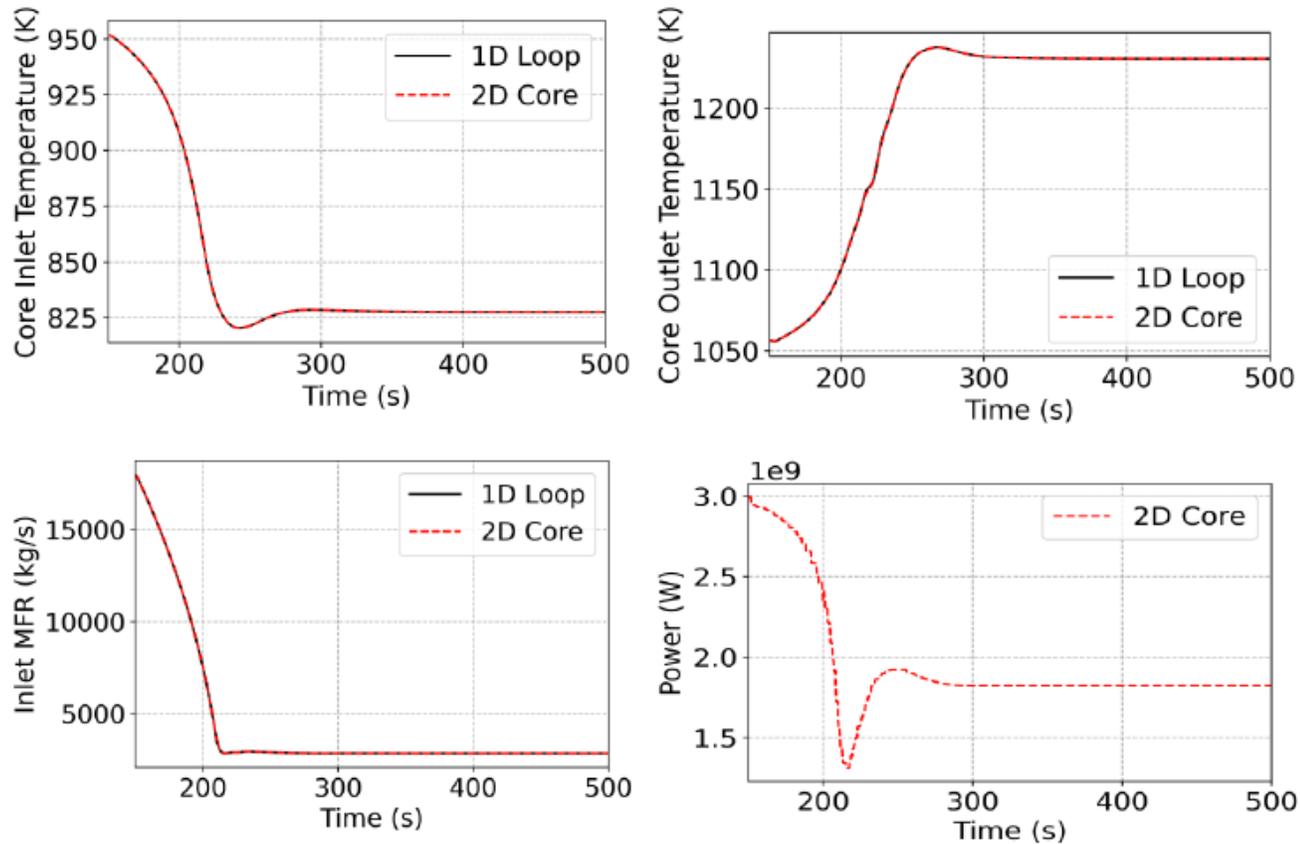


1D MSFR loop model



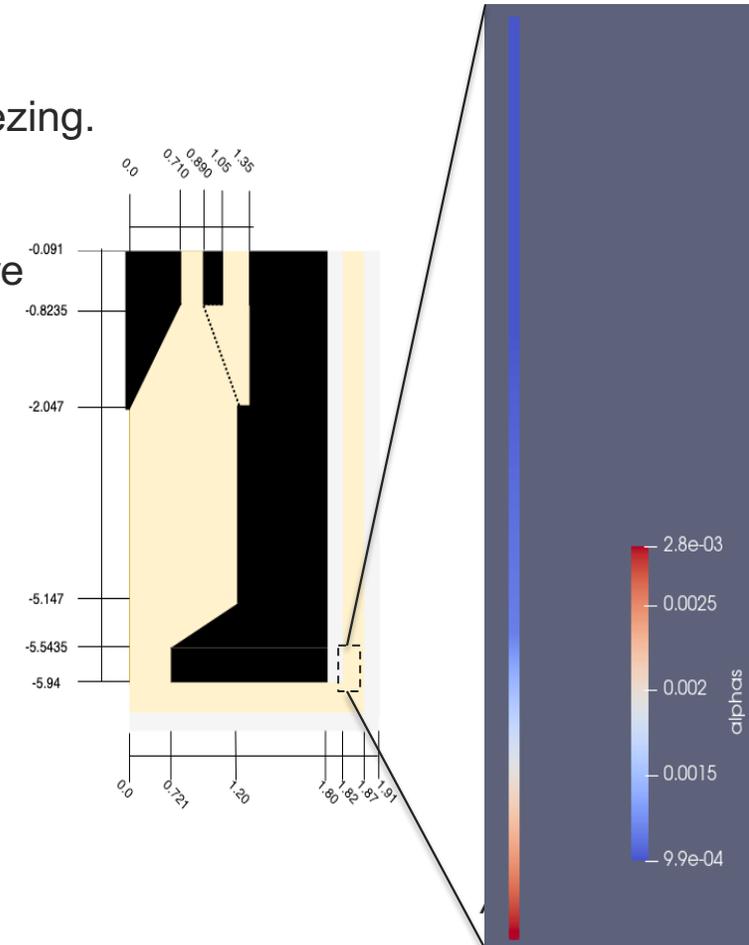
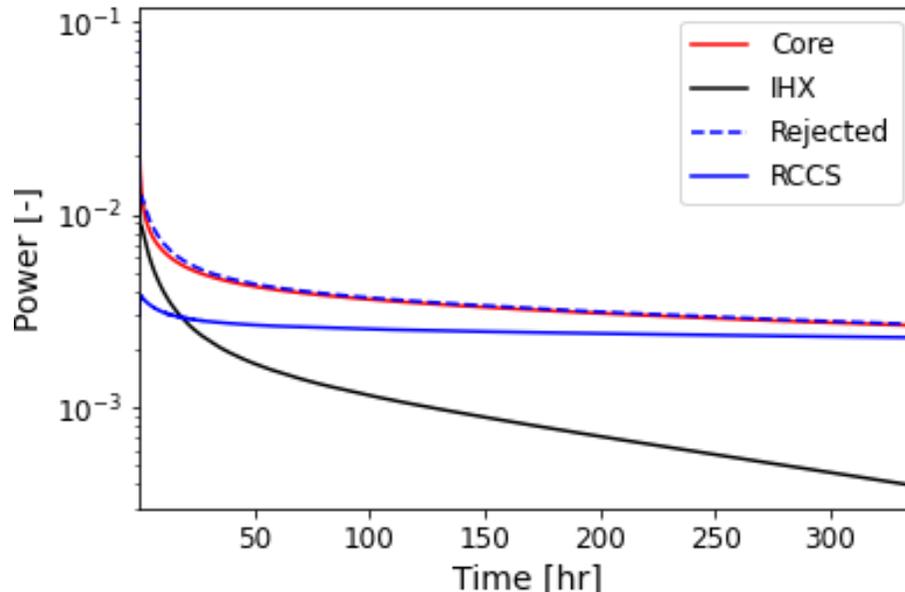
TRANSIENT SIMULATION RESULTS – PUMP TRIP

- Primary loop pump tripped, but the intermediate loop remains working.



SALT FREEZING MODEL

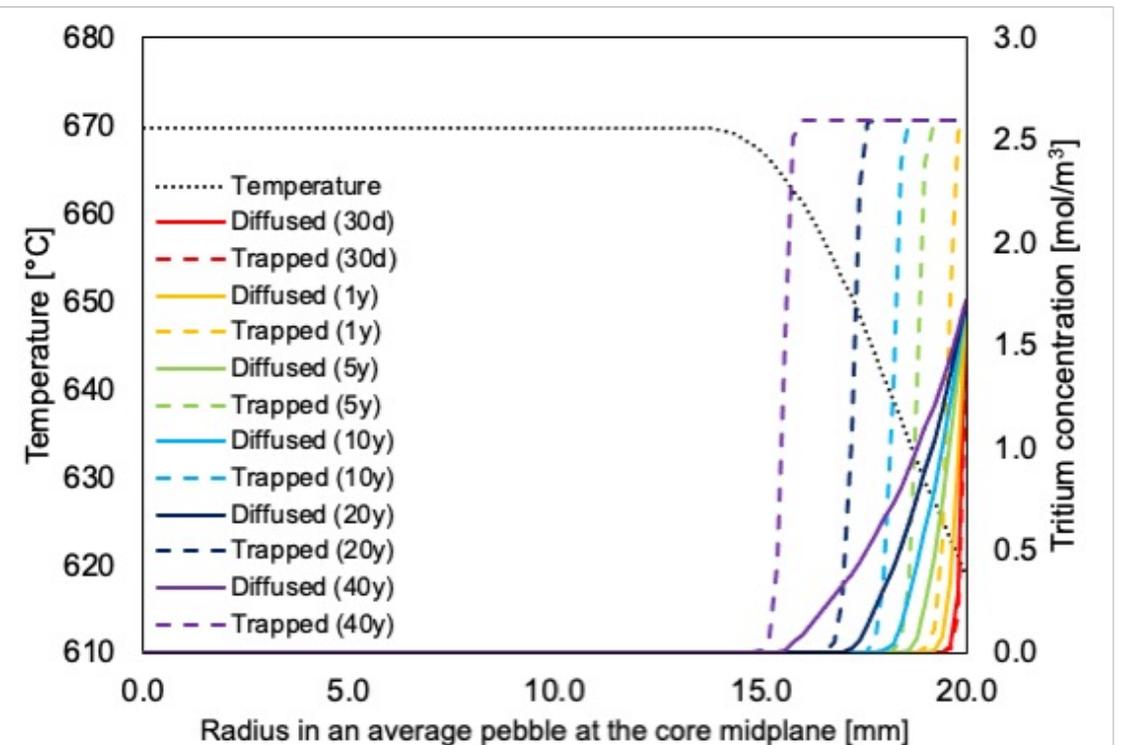
- Salt freezing may block key flow passages during overcooling transients.
- Initial model developed by an NEUP award;
- Further development in collaboration with Kairos Power
 - Typical geometries: plate, tube, annulus
 - Liquid-phase equations consider area varying effect
 - Solid-phase equations consider heat and mass balances associated with salt freezing.
- PBFHR overcooling simulation
 - PCP is lost, IHX remains operating, reactor protection system shuts down the core
 - System operates for ~14 days under natural circulation



TRITIUM TRANSPORT MODEL

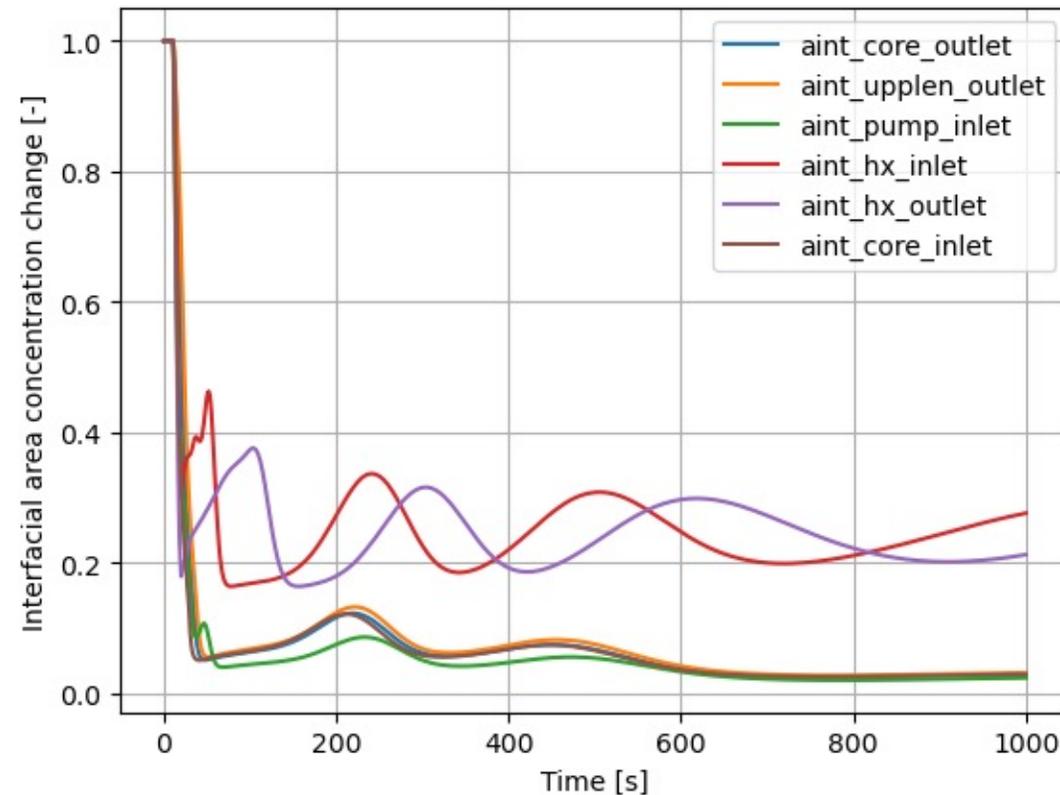
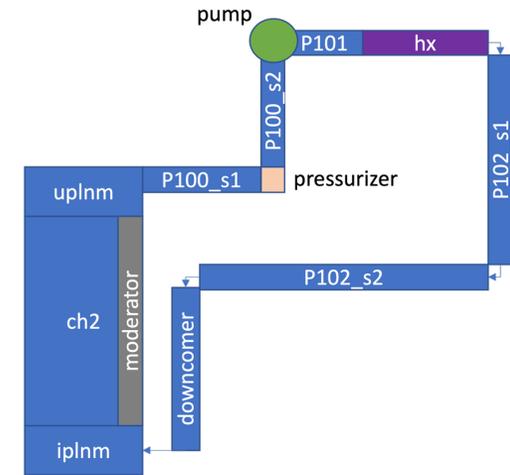
- System-level source term modeling is needed to capture a range of tritium transport phenomena:
 - Species generation and transport in fluid system
 - Uptake in graphite fuel pebbles and reflector
 - Permeation through pipe and vessel walls
- Species transport capability tightly coupled with thermal hydraulics:
 - Passive transport in 1D fluid components
 - Migration to fluid-solid interface via convection
 - Bulk diffusion through 2D solid components
 - Trapping & de-trapping within solid
 - Power-dependent generation rate in core
 - Models to capture evolution to cover gas
- Validation using Causey (1968) experiment

Tritium build-up in a reference FHR



GAS TRANSPORT MODEL

- Implementation of gas transport model into SAM
 - Focusing on bubble advection
- Includes modeling of drift velocity, calculation of volume fraction, bubble radius, and interfacial area
- Testing for unit verification and simple loop problems
- MSRE primary loop demonstration simulations
- Validation with Chavez (2021) experiment.



(Acknowledgements: R. Salko, ORNL)

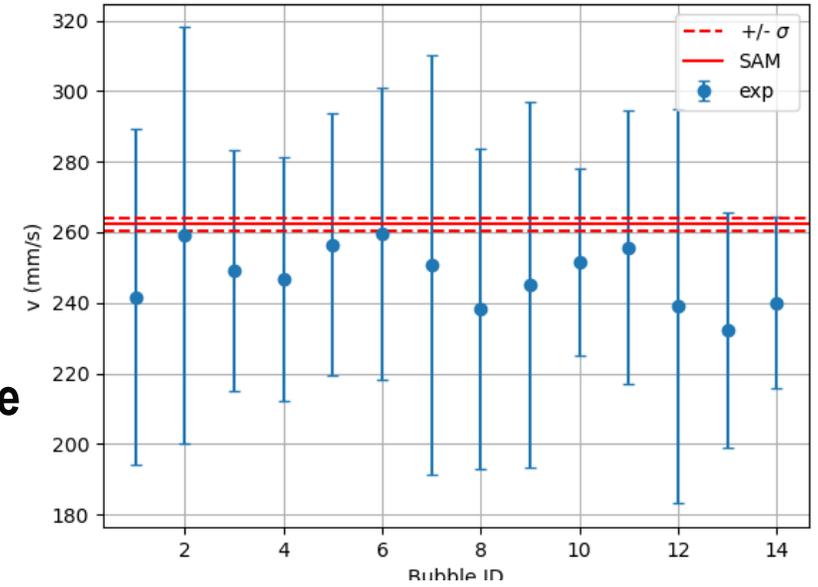
(R. Hu, J. Fang, D. Nunez, M. Tano, G. Giudicelli, R. Salko, ANL/NSE-22/56, 2022)

GAS MODEL VALIDATION

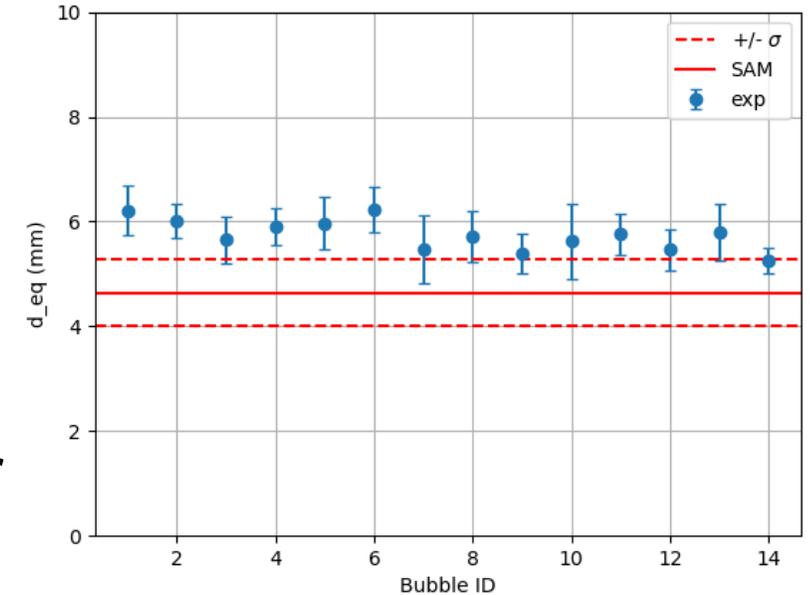
Denise E. Chavez, Se Ro Yang, Rodolfo Vaghetto, and Yassin Hassan. Experimental investigation of single helium bubbles rising in flinak molten salt. *International Journal of Heat and Fluid Flow*, 92, 2021. doi: 10.1016/j.ijheatfluidflow.2021.108875.

- Chavez tests measure helium bubbles rising in stagnant molten salt
- Dakota used to propagate measurement and fluid property uncertainty in SAM
- Overprediction of diameter likely related to wall effects in facility

Bubble rise velocity



Bubble diameter



CONCLUSION REMARKS

- SAM is a modern system-level analysis code for advanced non-LWR safety analysis, utilizing modern programming architecture, advanced numerical methods, and improved physical models.
- Complex and integrated code built upon simplest models aiming for simulating whole system behavior under simultaneous occurrence of various phenomena and processes.
- Preliminary MSR modeling capabilities assessed with code validations and demonstrations
 - System thermal fluids + PKE, including salt freezing
 - Species transport model, DNP, tritium, bubble transport
- Ongoing work
 - Continued physical model developments and usability improvements
 - RPI NEUP on mass transport; interface Saline for MSTDB-TB;
 - Continued V&V efforts, benchmark with MSRE and other IET tests
 - MSTTE at INL
 - Multi-scale multi-physics simulation of reference advanced reactor designs
 - Integrated simulations, as necessary, with Griffin, Mole, Nek/Pronghorn, Thermochemical, MSTDB

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ORNL: R. Salko

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THANK YOU!

QUESTIONS?

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